

【外部発表論文】冷却材喪失事故(LOCA)と被覆管に関する成果リスト

No	登録番号	著者氏名	原標題	資料名	発行年月	巻	号	開始ページ	終了ページ
1	A19711238	安藤良夫;三島良績;都甲泰正;川崎正之; 村主進;竹越尹	A Review of safety aspects of nuclear power plants in Japan	Proc.Fourth Int.Conf.Peaceful Uses Atomic Energy, 3	1972/00	3		279	295
2	A19733411	古田照夫;川崎了	Stress-rupture properties of helium-injected zircaloy-4	Journal of Nuclear Science and Technology	1974/06	11	6	268	269
3	A19744205	村主進;川崎了	原子炉安全工学講座,12; 冷却材喪失事故時の燃料の挙動	原子力工業	1974/08	20	8	70	74
4	A19766447	下桶敬則;松本潔	Probability distributions of peak-clad temperature and cladding oxidation thickness in loss-of-coolant accidents for a typical boiling water reactor	Nucl.Technol.	1977/01	35	1	119	130
5	A19777244	古田照夫;川崎了;橋本政男;大友隆	Oxygen embrittlement for zircaloy cladding	Journal of Nuclear Science and Technology	1978/02	15	2	152	155
6	A19799049	村尾良夫	Analytical study of thermo-hydrodynamic behavior of reflood-phase during LOCA	Journal of Nuclear Science and Technology	1979/00	16	11	802	817
7	A19799048	原山泰雄;古田照夫	ジルカロイ被覆管からみた合金設計への期待	材料科学	1979/00	16	4,5	173	179
8	A19800172	古田照夫;本橋治彦	Products at the surface of zircaloy cladding under a LOCA condition	J.Nucl.Mater.	1980/00	95	3	303	306
9	A19799215	古田照夫;川崎了	Acceleration of zircaloy-steam reaction rate by deformation under high temperature transient	Journal of Nuclear Science and Technology	1980/00	17	3	243	245
10	A19799097	鈴木元衛;川崎了	Development of computer code PRECIP-II for calculation of Zr-steam reaction; Comparison of calculation with experiments in temperature transient	Journal of Nuclear Science and Technology	1980/00	17	4	291	300
11	A19811260	鈴木元衛	High temperature deformation behavior of gradually pressurized zircaloy-4 tubes	Journal of Nuclear Science and Technology	1981/00	18	8	617	628
12	A19811063	石川迪夫;吉村富雄;星蔵雄;稻辺 輝雄	軽水炉における事故時の燃料破損形態と炉心冷却性	日本原子力学会誌	1981/00	23	12	930	938
13	A19800227	古田照夫;上塚寛;川崎了	Ductility loss of zircaloy cladding by inner-surface oxidation during high temperature transient	Journal of Nuclear Science and Technology	1981/00	18	10	802	810
14	A19800173	上塚寛;古田照夫;川崎了	Zircaloy-4 cladding embrittlement due to inner surface oxidation under simulated LOCA condition	Journal of Nuclear Science and Technology	1981/00	18	9	705	717
15	A19811362	古田照夫;川崎了	Reaction behavior of zircaloy-4 in steam-hydrogen mixture at high temperature	J.Nucl.Mater.	1982/00	105	2	119	131
16	A19811215	田坂 完二;斯波正誼;小泉 安郎;安濃田 良成;安部信明;鈴木光弘	ROSA-III base test series for a large break loss-of-coolant accident of boiling water reactor	Nucl.Technol.	1982/00	57		179	191

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17	A19811176	斎藤伸三;石島清見;塩沢周策;岩田耕司	Effects of rod pre-pressurization on light water reactor fuel behavior during reactivity accident conditions	Journal of Nuclear Science and Technology	1982/00	19	4	289	306
18	A19811169	上塚寛;古田照夫;川崎了	Embrittlement of Zircaloy-4 due to oxidation in environment of stagnant	Journal of Nuclear Science and Technology	1982/00	19	2	158	165
19	A19822383	鈴木元衛	Simulation test on interactions of zircaloy claddings deforming in fuel assembly under LOCA conditions	Journal of Nuclear Science and Technology	1983/00	20	6	475	490
20	A19822319	杉本純;村尾良夫	Experimental study of effect of initial clad temperature on reflood phenomena during PWR-LOCA	Journal of Nuclear Science and Technology	1983/00	20	8	656	667
21	A19844030	古田照夫;上塚寛;川崎了	Estimation of conservatism of present embrittlement criteria for zircaloy fuel cladding under LOCA	Zirconium in the Nuclear Industry	1984/00			734	746
22	A19844138	熊丸博滋;小泉 安郎;田坂 完二	Five percent break BWR LOCA/ECC test at ROSA-III without HPCS actuation; Two dimensional core thermal-hydraulic phenomena	Nucl.Eng.Des.	1985/00	86		219	239
23	A19833532	小泉 安郎;入子真規;与能本泰介;田坂 完二	Experiment analysis of power curve sensitivity test series at ROSA-III	Nucl.Eng.Des.	1985/00	86		267	287
24	A19855506	石田紀久;原山泰雄;知新之亮	Oxidation of 304 stainless steel in high-temperature steam	J.Nucl.Mater.	1986/00	140		74	84
25	A19855083	杉本純;須藤高史;村尾良夫	Analytical study of thermal response similarity between simulated fuel rods and nuclear fuel rods during reflood phase of PWR-LOCA	Journal of Nuclear Science and Technology	1986/00	23	4	315	325
26	A19855044	岩村公道;安達公道;傍島真	Experimental study of two-dimensional thermal-hydraulic behavior in core during reflood phase of PWR LOCA	Journal of Nuclear Science and Technology	1986/00	23	2	123	135
27	A19844391	与能本泰介;小泉 安郎;田坂 完二	Investigation of BWR LOCA at ROSA-III; Effect of break configuration on system transients	Nucl.Eng.Des.	1986/00	92		195	205
28	A19866245	井口正;村尾良夫;秋本肇	Assessment of core radial power profile effect model of REFLA code by using CCTF data	Journal of Nuclear Science and Technology	1987/07	24	7	536	546
29	A19866459	井口正;村尾良夫;杉本純;秋本肇;大久保努;北條恒行	Assessment of current safety evaluation analysis on reflood behavior during PWR-LOCA by using CCTF data	Journal of Nuclear Science and Technology	1987/11	24	11	887	896
30	A19880353	早田邦久	工学的安全研究	プロメテウス	1988/00	0	67	38	46
31	A19866465	中村秀夫;久木田豊;田坂 完二	BWR loss-of-coolant accident tests at ROSA-III with high temperature emergency core coolant injection	Journal of Nuclear Science and Technology	1988/02	25	2	169	179
32	B19901203	更田豊志;藤城俊夫	Hydrogen generation during cladding/coolant interactions under reactivity initiated accident conditions	Proc. of the 1st JSME/ASME Joint Int. Conf. on Nuclear Engineering	1991/00			271	277

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33	B19950189	秋本肇;大貫晃;村尾良夫	Assessment of predictive capability of REFLA/TRAC code for peak clad temperature during reflood in LBLOCA of PWR with small scale test, SCTF and CCTF data	Validation of Systems Transients Analysis Codes (FED-Vol. 223)	1995/00	0		8	
34	B19941281	秋本肇;大貫晃;村尾良夫	Assessment of REFLA/TRAC code for system behavior during reflood phase in a PWR LOCA with CCTF data	Proc. of the 2nd Int. Conf. on Multiphase Flow 95-Kyoto	1995/00	0		P2¥_37	P2¥_44
35	B19961423	大貫晃;秋本肇	Assessment of REFLA/TRAC code for heat transfer enhancement phenomena during the reflood phase of PWR-LOCA	Proc. of 5th Int. Topical Meeting on Nuclear Thermal Hydraulics, Operations and Safety	1997/04	00	00	1	6
36	A19980158	大久保 忠恒*;何川 修一*;鈴木 元衛	ハルデン拡大計画会議	核燃料	1998/00		29	12	13
37	A19980160	浅香英明;安濃田 良成;久木田豊;大津巖	Secondary-side depressurization during PWR cold-leg small break LOCAs based on ROSA-V/LSTF experiments and	Journal of Nuclear Science and Technology	1998/12	35	12	905	915
38	B19991754	更田 豊志;永瀬 文久;中村 武彦;笛島 栄夫;上塚 寛	JAERI research on fuel rod behavior during accident conditions	NUREG/CP-0169	1999/10			341	354
39	B19992050	永瀬 文久;大友 隆;谷本 政隆;上塚 寛	Experiments on high burnup fuel behavior under LOCA conditions at JAERI	Proceedings of the 2000 International Topical Meeting on LWR Fuel Performance (CD-ROM)	2000/04			15	
40	A20000469	永瀬 文久	LOCA時挙動	最新核燃料工学;高度化の現状と展望	2001/06			148	155
41	B20011397	永瀬 文久;上塚 寛	Study of high burnup fuel behavior under LOCA conditions at JAERI; Hydrogen effects on the failure-bearing capability of cladding tubes	NUREG/CP-0176	2002/05			335	342
42	B20010281	永瀬 文久;谷本 政隆;上塚 寛	Study on high burnup fuel behaviour under a LOCA conditions at JAERI	IAEA-TECDOC-1320	2002/11			270	278
43	A20020496	永瀬 文久;大友 隆;上塚 寛	Oxidation kinetics of low-Sn Zircaloy-4 at the temperature range from 773 to 1573 K	Journal of Nuclear Science and Technology	2003/04	40	4	213	219
44	A20020020	安田 良;仲田 祐仁;松林政仁;原田 克也;畠山 祐一;天野 英俊	Application of hydrogen analysis by neutron imaging plate method to Zircaloy cladding tubes	Journal of Nuclear Materials	2003/08	320	3	223	230
45	A20030569	永瀬 文久;更田 豊志	Recent results from LOCA study at JAERI	NUREG/CP-0185	2004/00			321	331
46	B20040082	杉山 智之;永瀬 文久;中村 仁一;更田 豊志	LOCA and RIA studies at JAERI	HPR-362, Vol.2	2004/05			12	
47	A20030577	永瀬 文久;更田 豊志	Effect of pre-hydriding on thermal shock resistance of Zircaloy-4 cladding under simulated loss-of-coolant accident conditions	Journal of Nuclear Science and Technology	2004/07	41	7	723	730
48	B20040678	永瀬 文久;更田 豊志	Results from simulated LOCA experiments with high burnup PWR fuel claddings	Proceedings of 2004 International Meeting on LWR Fuel Performance	2004/09			500	506
49	A20030578	鈴木 元衛;草ヶ谷 和幸*;斎藤 裕明*;更田 豊志	Analysis on lift-off experiment in Halden reactor by FEMAXI-6 code	Journal of Nuclear Materials	2004/12	335	3	417	424

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50	09-2004-0233	山下 真一郎;吉武 康光;赤坂 尚昭;鵜飼 重治;大貫 惣明	Nano-meso Structures and Ring-tensile Properties of Neutron-irradiated ODS	Vol.475-479(2005)1467	2005/00			0	
51	A20040146	永瀬 文久;更田 豊志	Behavior of pre-hydrided Zircaloy-4 cladding under simulated LOCA conditions	Journal of Nuclear Science and Technology	2005/02	42	2	209	218
52	B20051177	更田 豊志;永瀬 文久;杉山 智之	RIA- and LOCA-simulating experiments on high burnup LWR fuels	Proceedings of IAEA Technical Meeting on Fuel Behaviour Modelling under Normal, Transient and Accident Conditions, and High Burnups (CD-ROM)	2005/09			15	
53	BB20050127	中村 武彦;更田 豊志;永瀬 文久;杉山 智之	Behavior of high burnup fuels under RIA and LOCA conditions	HPR-364, Vol.1	2005/10			16	
54	B20051174	永瀬 文久;更田 豊志	Embrittlement and fracture behavior of pre-hydrided cladding under LOCA conditions	Proceedings of 2005 Water Reactor Fuel Performance Meeting (CD-ROM)	2005/10			668	677
55	B20050844	鈴木 元衛;斎藤 裕明*;更田 豊志	RANNS code analysis on the local mechanical conditions of cladding of high burnup fuel rods under PCMI in RIA-simulated experiments in NSRR	Proceedings of 2005 Water Reactor Fuel Performance Meeting (CD-ROM)	2005/10			579	601
56	A20040624	永瀬 文久;更田 豊志	Results from studies on high burn-up fuel behavior under LOCA conditions	NUREG/CP-0192	2005/10			197	230
57	BB20050006	小野澤 淳;原田 晃男;本田 順一;安田 良;仲田 祐仁;金沢 浩之;西野 泰治	Improved technique of hydrogen concentration measurement in fuel cladding by backscattered electron image	JAEA-Conf 2006-003	2006/05			212	221
58	AA20050380	永瀬 文久;更田 豊志	Fracture behavior of irradiated Zircaloy-4 cladding under simulated LOCA conditions	Journal of Nuclear Science and Technology	2006/09	43	9	1114	1119
59	AA20050163	鈴木 元衛;斎藤 裕明*;更田 豊志	RANNS code analysis on the local mechanical conditions of cladding of high burnup fuel rods under PCMI in RIA-simulated experiments in NSRR	Journal of Nuclear Science and Technology	2006/09	43	9	1097	1104
60	A20050068	鈴木 元衛;更田 豊志;斎藤 裕明*	Analysis on pellet-clad mechanical interaction process of high-burnup PWR fuel rods by RANNS code in Reactivity-initiated accident conditions	Nuclear Technology	2006/09	155	3	282	292
61	BB20060352	山野 秀将;飛田 吉春	An Evaluation of CDA event sequences in a gas-cooled fast reactor	Proceedings of 5th Korea-Japan Symposium on Nuclear Thermal Hydraulics and Safety (NTHAS-5)	2006/11			716	723
62	AA20070357	梅田 幹	燃料安全研究国際会議2007(Fuel Safety Research Meeting2007);軽水炉燃料の事故時挙動評価に関する研究成果の報告の	日本原子力学会核燃料部会ホームページ(インターネット)	2007/06			2	
63	BB20071974	竹田 武司;浅香 英明;鈴木 光弘;中村 秀夫	RELAP5 analysis of ROSA/LSTF vessel upper head break LOCA experiment	Proceedings of 12th International Topical Meeting on Nuclear Reactor Thermal Hydraulics (NURETH-12) (CD-ROM)	2007/09			3	

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64	BB20081681	小野澤 淳;原田 晃男;本田 順一;仲田 祐仁;金沢 浩之;佐川 民雄	Improved technique for hydrogen concentration measurement in fuel claddings by backscattered electron image analysis, 2	JAEA-Conf 2008-010	2008/12			325	332
65	AA20080858	中頭 利則;永瀬 文久;更田 豊志	High temperature oxidation of Nb-containing Zr alloy cladding in LOCA	Nuclear Engineering and Technology	2009/03	41	2	163	170
66	BB20090522	大貫 惣明*;橋本 直幸*;鵜飼 重治*;木村 晃彦*;井上 賢紀;皆藤 威二;藤澤 敏治*;奥田 隆成*;阿部 富士雄*	Super ODS steels R&D for fuel cladding of next generation nuclear systems, 9; Damage structure evolution under electron-irradiation	Proceedings of 2009 International Congress on Advances in Nuclear Power Plants (ICAPP '09) (CD-ROM)	2009/05			9307Y_1	9307Y_4
67	BB20083296	大平 博昭;高松 操	Draft; Thermal-hydraulic evaluation of Joyo fuel subassembly with local blockage	Proceedings of 17th International Conference on Nuclear Engineering (ICONE-17) (CD-ROM)	2009/06			8	
68	AA20080754	永瀬 文久;中頭 利則;更田 豊志	Behavior of high burn-up fuel cladding under LOCA conditions	Journal of Nuclear Science and Technology	2009/07	46	7	763	769
69	BB20083574	鈴木 元衛;杉山 智之;宇田川 豊;永瀬 文久;更田 豊志	Comparative analysis on behavior of high burnup PWR fuels pulse-irradiated in reactivity-initiated accident conditions	Proceedings of Top Fuel 2009 (DVD-ROM)	2009/09			473	479
70	BB20083576	永瀬 文久;中頭 利則;更田 豊志	Cladding embrittlement under LOCA conditions, examined by two test methodologies	Proceedings of Top Fuel 2009 (DVD-ROM)	2009/09			527	537
71	BB20101067	柳澤 和章	Failure and its mechanism of LWR and research reactor fuels	Proceedings of 3rd International Symposium on Symbiotic Nuclear Power Systems for 21st Century (ISSNP 2010) (CD-ROM)	2010/08			10	
72	AA20100562	柳澤 和章	Failure and its mechanism of LWR and research reactor fuels	International Journal of Nuclear Safety and Simulation	2010/09	1	3	246	257
73	BB20102395	中塚 亨;三澤 丈治;吉田 啓之;高瀬 和之	Numerical analysis on thermal-hydraulics of supercritical water flowing in a tight-lattice fuel bundle	Proceedings of Joint International Conference of 7th Supercomputing in Nuclear Application and the 3rd Monte Carlo (SNA + MC 2010) (USB Flash Drive)	2010/10			4	